1013 2024 Fall Meeting

Study of changes in melting behaviour of the UO<sub>2</sub>-Zircaloy-2 system induced by addition of magnetite at temperatures 2000 and 2100 °C

\*Bella Zubekhina, Masahide Takano, Anton Pshenichnikov Japan Atomic Energy Agency

**Abstract:** Interactions in two systems UO<sub>2</sub>-Zircaloy and UO<sub>2</sub>-Zircaloy-magnetite were studied at temperatures 2000 and 2100 °C during 30 min in argon atmosphere. It was shown that presence of relatively small amount of magnetite in UO<sub>2</sub>-Zircaloy system suppresses UO<sub>2</sub> dissolution in Zr melt and affects the composition of new-formed phases.

Keywords: fuel, cladding, interaction, corium, magnetite

## Introduction

An interaction between UO<sub>2</sub> fuel and Zircaloy (Zry) cladding is a well-known process taking place during severe nuclear accident with core melting. In case of the 1F accident, due to considerable amount of stainless steel one could expect the influence of steel and steel oxidation products to the process of fuel debris formation [1]. This paper supposed to improve the understanding of UO<sub>2</sub>-Zry-Fe<sub>3</sub>O<sub>4</sub> system behavior during melting at temperatures 2000-2100 °C.

## Results and discussion

A possibility of U diffusion from oxide phase to metallic melt was studied at temperatures 2000 and 2100 °C in argon atmosphere. The molar ratios  $UO_2$ : Zry and  $UO_2$ : Zry:  $Fe_3O_4$  were 15:1 and 15:1:0.15, respectively. Despite the low molar ratio of  $Fe_3O_4/Zry$ , after 30 min of interaction the average bulk content of U dissolved in the melt was about 6 times lower for  $UO_2$ -Zry- $Fe_3O_4$  system than for  $UO_2$ -Zry. At both temperatures, 2000 and 2100 °C, main phases after  $UO_2$ -Zry- $Fe_3O_4$  interaction were Fe-Zr-O,  $ZrO_2$ ,  $\alpha$ -Zr(O), and Zr-O-O with U content varied from 1 at.% to 20 at.%

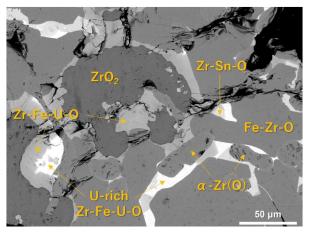


Figure 1. Components of the melt after reaction at 2100 °C.

(Figure 1). When Fe<sub>3</sub>O<sub>4</sub> reacted together with Zry, the volume fraction of the typical corium phase  $(U,Zr)O_2$  became almost negligible. Several minor inclusions of  $(U,Zr)O_2$  phase were found in the local area of the sample. System  $UO_2$ -Zry at temperatures 2000 and 2100 °C showed formation of U-Zr-O solid solutions of various composition and  $\alpha$ -Zr(O) phase which is in a good agreement with previously published data [2].

## Conclusion

The results obtained allow suggesting that presence of iron oxide may suppress the UO<sub>2</sub> dissolution in a Zry melt during severe accident progression and significantly affects the phase composition of new-formed fuel debris.

## References

- [1] Barrachin, M. Corium Experimental Thermodynamics: A Review and Some Perspectives. Thermo 2021, 1, 179-204.
- [2] Hofmann, P. Current knowledge on core degradation phenomena, a review. J. Nuc. Mat., 1999, 270, 1–2, 194-211.